原規規発第 22101910 号 令和 4 年 10 月 19 日

国立研究開発法人日本原子力研究開発機構 理事長 小口 正範 殿

原子力規制委員会

核燃料輸送物設計承認英文証明書について

核燃料物質等の工場又は事業所の外における運搬に係る核燃料輸送物設計承認及び容器 承認等に関する申請手続ガイド(令和2年2月26日付け原規規発第2002264号)2.4.に基 づき、令和4年10月13日付け令04原機(環材)028をもって申請のあった標記の件につい て、添付のとおり証明します。

IDENTIFICATION MARK J/2044/B(U)F

$\begin{array}{c} \text{COMPETENT AUTHORITY} \\ \text{OF} \\ \text{JAPAN} \end{array}$

CERTIFICATE FOR APPROVAL OF PACKAGE DESIGN FOR THE TRANSPORT OF RADIOACTIVE MATERIALS

ISSUED BY

NUCLEAR REGULATION AUTHORITY 1-9-9, ROPPONGI MINATO-KU TOKYO, JAPAN

CERTIFICATE FOR APPROVAL OF PACKAGE DESIGN FOR THE TRANSPORT OF RADIOACTIVE MATERIALS

This is to certify, in response to the application by Japan Atomic Energy Agency, that the package design described herein complies with the design requirements for a package containing Medium Enriched Uranium Fuels(Spent Fuel Elements) and Low Enriched Uranium Fuels(Spent Fuel Elements), specified in the 2018 Edition of the Regulations for the Safe Transport of Radioactive Material (International Atomic Energy Agency, Safety Standards Series No.SSR-6) and the Japanese rules based on the Act on Regulation of Nuclear Source Material, Nuclear Fuel Material and Reactors.

This certificate does not relieve the consignor from compliance with any requirement of the government of any country through or into which the package will be transported.

COMPETENT AUTHORITY
IDENTIFICATION MARK: J/2044/B(U)F

Oct / 19/2022

Hasegawa Kiyomitsu

Director, Division of Licensing for Nuclear Fuel Facilities

Secretariat of Nuclear Regulation Authority Competent Authority of JAPAN for Package Design Approval

1. The Compete	ent Authority Identificatio	n Mark : J/2044/B(U)F
2. Name of Pac	kage	: JMS-87Y-18.5T
3. Type of Pack	age	: Type B(U) package for fissile material
4. Specification	of Package	
(1) Materials o	f Packaging	
(a) Boo	dy and Lid	: Stainless steel
(b) Bas	sket	: Stainless steel and
(c) Sho	ock absorber	: Stainless steel and
(2) Total Weight (3) Outer Dime	ht of Packaging ensions of Packaging	: 18,110 kg or less
(i) Outer Dia	meter	: Approximately 1,900 mm
(ii) Height		: Approximately 2,000 mm
(4) Total Weigl	ht of Package	: 18,440 kg or less
(5) Illustration	of Package	: See Figure-1
		(Cutaway view)
5. Specification	of Radioactive Contents	: See Table-1
6. Description of	of Containment System	•
-	t system consists of the bo	dy, the lid, the vent plug and
the drain val	ve.	
	is used for contact surfa	ace of body and lid, body and vent plug, body and
drain valve a	and the valve seat.	
7. For Package	Containing Fissile Materi	als,
(1) Restrictions	-	
(i) Restriction	n Number "N"	: No restriction
(ii) Array of F	ackage ackage	: No restriction
(iii) Criticality	Safety Index (CSI)	:0
(2) Description	of Confinement System	
Confineme	nt system consists of the b	asket which maintains the fuel elements contained
in the pack	age.	
(3) Assumption	ns of Leakage of Water int	o Package
It is assume	ed in criticality analysis th	nat water will leak into void spaces of inner
packaging.		•
(4) Special Fea	tures in Criticality Assess	ment
Not applie	able	

8. For Type B (M) Packages, a Statement Regarding Prescriptions of Type B (U) Package that do not apply to this Package

Not applicable (This package is Type B(U))

9. Assumed Ambient Conditions

(i) Ambient Temperature Range

:-40°C~38°C

(ii) Insolation Data

: Table 12 of IAEA Regulation

- 10. Handling, Inspection and Maintenance
 - (1) Handling Instructions
 - Package should be handled carefully in accordance with the schedule and procedures established properly taking all possible safety measures.
 - (ii) Package should be handled using appropriate lifting devices and the crane.
 - (iii) When packaging is stored outdoors, it should be covered with an appropriate waterproof sheet, avoiding the situation where it is placed directly on the ground.
- (2) Inspections and Maintenance of Packaging

 The following inspections should be performed not less than once a year (once for every ten times in a case where the packaging is used more than ten times a year) and defect of packaging should be repaired, if any, in order to maintain the integrity of packaging.
 - (i) Visual Inspection
 - (ii) Pressure Measurement Inspection
 - (iii) Leak Tightness Inspection
 - (iv) Maintenance of O-rings, Valve, etc., used for Containment System
 - (v) Shielding Inspection
 - (vi) Subcriticality Inspection
 - (vii) Heat Transfer Inspection
 - (viii)Lifting Inspection
- (3) Actions Prior to Shipment

The following inspections should be performed prior to shipment.

- (i) Visual Inspection
- (ii) Lifting Inspection
- (iii) Weight Inspection
- (iv) Surface Contamination Inspection
- (v) Dose Rate Inspection
- (vi) Subcriticality Inspection
- (vii) Contents Inspection
- (viii) Surface Temperature Inspection
- (ix) Leak Tightness Inspection
- (x) Pressure Inspection
- (4) Precautions for Loading of Package for Shipment

Package should be securely loaded to the conveyance at the designated tie down portion of the packaging so as not to move, roll down or fall down from the loading position during transport.

11. Issue Date and Expiry Date

(i) Issue Date

: Sep 21, 2022

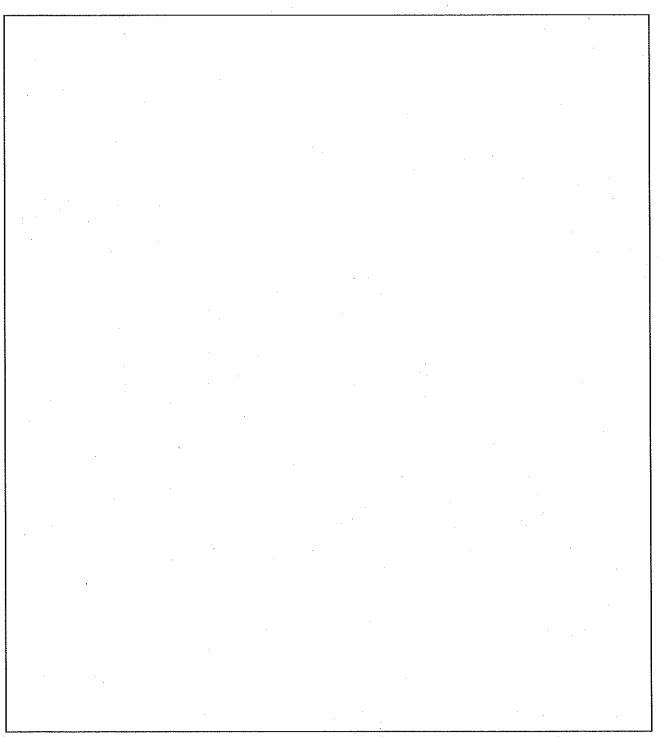
(ii) Expiry Date

Sep 20, 2062

However, if this certificate no longer meets the technical standards (limited to those related to the design of package) due to a revision of the regulations*1,2, this certificate will be expired.

*1 The NRA Ordinance on Off-Site Transportation of Nuclear Fuel Materials, etc. (Ministerial ordinance issued by the Prime Minister's Office No. 57 of 1978)

*2 The Notification on Technical Details for Off-Site Transportation of Nuclear Fuel Materials, etc. (Notice issued by Science and Technology Agency No. 5 of 1990)



 $\underline{Figure \hbox{-}1\ Illustration\ of\ JMS\hbox{-}87Y\hbox{-}18.5T\ Package\ (Cutaway\ view)}}$

Table-1 Specification of Radioactive Contents

		_										····				TCTC:		Page 5 of 5 Pag	ges
Low Enriched Uranium Fuels (LEU) (Spent Fuel Elements)	JMTR Standard Fuel JMTR Follower Fuel Elements (LEU)	JMTR	Plate Fuel	30 or less											Uranium-Silicon-Aluminum Dispersion Alloy	Aluminum Alloy			
									2011.1	Solid							1.88	of "Spent Fuel Elements") or mixed fuels (i.e. two or more kinds of "Spent Fuel Elements") rface of the package is 1 Gy/h or more. ere will be no additional spent fuel in the future, the minimum number of cooling time for 22, the radioactivity intensity has decreased by about	•
					:										Uranium-Silicon-4	Alu	2.40	or mixed fuels (i.e. two y/h or more. to fuel in the future, sity has decreased by s	
														i de de proprieta de la companya de			2.80	it Fuel Elements"). the package is 1 G be no additional sp adioactivity intens	
Medium Enriched Uranium Fuels (MEU)(Spent Fuel Elements)	JMTR Standard Fuel Elements (MEU)	JMTR	Plate Fuel	30 or less					C-:[-0	Solid					Uranium Aluminum Dispersion Alloy	Aluminum Alloy	1.98	Each package shall be loaded with single fuel (i.e. only one kind of "Spent Fuel Elements") or mixed fuels (i.e. two or more kinds of "Spent Fuel Elements") within an identical group. The absorbed dose rate to air at a position 1 m away from the surface of the package is 1 Gy/h or more. This is the period set for the design of the package, and since there will be no additional spent fuel in the future, the minimum number of cooling time for the fuel to be transported is	
Fuel Element		Reactor	Fuel Type	Number of Fuel Elements (elements/Package)	Enrichment (wt%)	235U weight	U weight (g /element)	Burn-up (%)	Cooling Time (days)	Condition	Total (TBq /30 elements)	Principal	Radionuclide	(Saliented oc. part)	Fuel Core	Cladding and Side Plate	Total Heat Generation Rate (kW/30 elements)	iach package shall be loaded with swithin an identical group. The absorbed dose rate to air at a positive is the period set for the design he fuel to be transported is	
	Type			Number (elen		Initial Value	7		Cooli			Activity of	Contents	`	1	Maceriai	Total He (kW	Each packag within an i The absorbed This is the puthe fuel to be	, •