

(Provisional Translation) **Measures for Mid-term Risk Reduction at TEPCO's Fukushima Daiichi NPS (as of March 2022)**

March 9, 2022  
Nuclear Regulation Authority Japan

Areas for Risk Reduction	Major Measures (Measures for 10 years from now)
Liquid Radioactive Material	<p><b>【Conditions to be realized】</b> Treat all the liquid radioactive material including those remaining in tanks</p> <ul style="list-style-type: none"> <li>•Progress the treatment of stagnant water containing α nuclides in buildings and maintain the buildings as drainage completed area except for reactor buildings.</li> <li>•Decrease rainwater and groundwater flowing into buildings to prevent the increase of stagnant water in buildings and complete the treatment of all stagnant water.</li> <li>•Decrease water in S/C of Unit 1 and 3 to the level at which the water will not leak out of the buildings</li> </ul>
Spent Fuel	<p><b>【Conditions to be realized】</b> Store all spent fuels in dry storage casks</p> <ul style="list-style-type: none"> <li>•Complete removing all fuels from spent fuel pools of each Unit</li> <li>•Expand dry storage cask area and secure the storage for all the spent fuels together with the capacity of common pool</li> <li>•Transfer fuels stored in common pool, into dry storage casks as early as possible</li> </ul>
Solid Radioactive Material	<p><b>【Conditions to be realized】</b> Stabilize high dose waste by processing such as dehydration</p> <ul style="list-style-type: none"> <li>•Removal of high-dose zeolite sandbags in Process Main Building and sludge from Decontamination Facility, and treatment to prevent scattering or leakage</li> <li>•Dehydration of slurry stored inside HIC (High Integrity Container)</li> </ul> <p><b>【Conditions to be realized】</b> Store and manage the waste appropriately depending on classification by radioactivity concentration and characteristics</p> <ul style="list-style-type: none"> <li>•Classify the materials generated from decommissioning process, e.g. dismantle of buildings, by radioactivity concentration and characteristics, and store and manage them properly</li> <li>•Store and manage spent Cesium adsorption vessel stably inside facilities</li> <li>•Proceed with volume reduction and incineration of solid waste such as rubble to reduce the amount of solid waste and dissolve temporary storage outside</li> </ul> <p><b>【Conditions to be realized】</b> Install analytical facility and strengthen analytical capacity to advance the decommissioning process</p> <ul style="list-style-type: none"> <li>•Install comprehensive analytical facility which can analyze wide variety and large amount of radioactive materials, and install analytical facility which is necessary to understand the characteristics of debris</li> <li>•Evaluate the needs of radioactive material analysis and ensure the staffs and their ability to conduct it certainly</li> </ul> <p><b>【Conditions to be realized】</b> Store fuel debris stably</p> <ul style="list-style-type: none"> <li>•Take safety measures in removing fuel debris and store debris in stable state</li> </ul>
Countermeasures for External Events	<ul style="list-style-type: none"> <li>•Seal outer wall of buildings and restrain inflow of groundwater into buildings significantly</li> <li>•Repair damaged parts such as building roof to prevent rainwater inflow</li> <li>•Take measures against deterioration and damage of building structures, etc.</li> </ul>
Important Areas to Progress Decommissioning	<ul style="list-style-type: none"> <li>•Reinforce organizational structure to progress risk reduction swiftly, and strengthen quality management</li> <li>•Reduce radiation doses by removal of high-dose radiation sources such as lower part of Exhaust stack of Unit 1 and 2 or shielding against them, and take measures for suppressing dust scattering during operation inside R/B</li> <li>•Discharge the ALPS treated water into the sea according to the plan</li> <li>•Consider the impact of the contamination beneath the shield plugs to each decommissioning work</li> </ul>

## Measures for Mid-term Risk Reduction at TEPCO's Fukushima Daiichi NPS (Main Goals)

Area Fiscal Year	Liquid Radioactive Material	Spent Fuel	Solid Radioactive Material		Countermeasures for External Events	Important Areas to Progress Decommissioning	
2022	Approach toward stopping water injection to reactor	Start fuel removal from Unit 6	Start operation of analytical facility "Laboratory-1"	Install volume reduction treatment facility	Widen the paving area around buildings 【against rainwater】 (to be completed in FY2023)	Remove high-dose SGTS pipes in lower part of exhaust stack of Unit 1 and 2	
	Approach to decrease the water level in S/C of Unit 1 and 3	Provide shielding in Unit2 R/B Operating Floor and suppress dust scattering (to be completed in FY2023)	Develop an analysis plan (include facility and human resources)	Investigate inside Unit 1 PCV		Install seismograph in Unit 1 and 2	Consider the impact of the contamination beneath the shield plugs to each decommissioning work
	Determine the treatment method of untreated water in tanks		Retrieve fuel debris from Unit 2 experimentally, investigate inside PCV and analyze debris				Improve workplace environment continuously
			Start installation of crane for large waste storage facility (Cs adsorption vessel)			Reinforce quality management structure of Decommissioning Project	
			Start installation of ALPS slurry (HIC) stabilization facility			Reduction of exposure under high-dose environment	
2023	Start treatment of untreated water in tanks		Start removal of Zeolite etc. in Process Main Building, etc.	Start removing Sludge from Decontamination Facility		Take measures to suppress dust scattering from buildings, etc.	
	Half the amount and treat stagnant water in R/B		Start operation of solid waste storage facility 10 (First half of 2023FY)			Start the ALPS treated water discharge into the sea	
			Safety measures for stepwise expansion of retrieval of fuel debris inside Unit 2				
			Install large waste storage facility (Cs adsorption vessel)				
2024		Install Unit 1 R/B cover	Install ALPS slurry (HIC) stabilization facility		Establish the evaluation method to check the soundness of Buildings		
		Start fuel removal from Unit 5					
Further Goals 2025 ~ 2033	Dry up Process Main Building, etc.	Expand dry storage cask area to install additional dry casks	Install analytical facility "Laboratory-2" and other fuel debris analytical facility		Seal outer wall of buildings 【against groundwater】		
	Treat all stagnant water in R/B	Fuel removal from Unit 1 and 2	Dissolve outside storage of rubbles	Store retrieved fuel debris in stable state			
		Fuel removal from spent fuel pool of all units	Control waste in safer and more stable state	Install comprehensive analytical facility			

Countermeasures for Risks which would have an effect on the human and the environment  
 Countermeasures for Risks which effect on offsite is relatively small, but still need attention

## Measures for Mid-term Risk Reduction at TEPCO's Fukushima Daiichi NPS (Other Tasks)

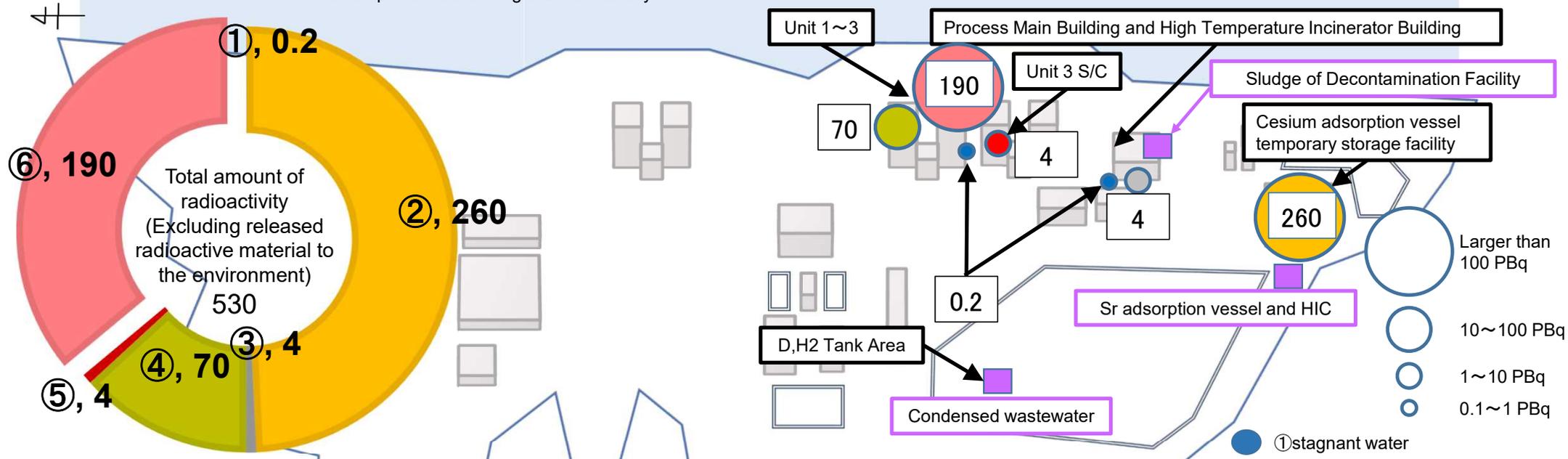
<input type="radio"/>		Timing
<input type="radio"/> Liquid Radioactive Material	To be conducted	Transfer ALPS slurry stored inside HIC to new HIC Within FY2023
		※Transfer 45 units of HIC, whose accumulated absorbed dose exceeded the limit(5,000 kGy) by the end of January 2022 Within FY2022
	Timing has not been decided	Remove underground cisterns Treatment of sludge etc. remaining in dried up buildings
<input type="radio"/> Spent Fuel		Timing
To be conducted	Begin to remove spent control rods	Within FY2022
<input type="radio"/> Solid Radioactive Material		Timing
To be conducted	Dissolve the temporary storage areas	Within FY2022
<input type="radio"/> Countermeasures for external events		Timing
	Restrain the inflow of rainwater into radioactive waste treatment buildings of Unit 1 and 2	Within FY2022
To be conducted	Expansion works of D drainage 【measures against heavy rains】	Within FY2022
	Install tide embankment against Nihon-trench Tsunami	Within FY2023

<input type="radio"/>		Timing
<input type="radio"/> Important issues to progress decommissioning	To be continued	Survey the contamination situation inside the reactor buildings, etc. (nuclide analysis, etc.) Understand the properties and characteristics of the cooling water after cooling the reactors down (nuclide analysis, etc.) Analyze the flow of contaminated water inside the reactor buildings, etc. Directly observe inside the containment vessel and pressure vessel ※Observation inside the pressure vessel is to be conducted afterwards
		Reduce concentration of radioactive materials in the water of drainages
	To be conducted	Investigation of other systems of Unit 3 and the other Units considering the case of accumulation of hydrogen in RHR system of Unit 3 Within FY2022
	Investigate contamination on the bottom and around Unit 1 and 2 common stack	Within FY2023
To be considered the necessity	Consider methods to improve the environment of ground level 2.5m, such as removal and decontamination of soil, purification of ground water, etc.	

# Location of radioactive materials (Mainly Cs-137) (except for spent fuels) (unit; PBq)

	type*	characteristic	Explanation for each type
①	Stagnant water	Liquid	Highly contaminated water stagnating in 1~3 Reactor Buildings, Process Main Building and High Temperature Incinerator Building
⑤	Unit 3 S/C	Liquid	Highly contaminated water in the Unit 3 S/C
③	Zeolite	Liquid/Solid	Precipitation from treatment of contaminated water soon after the accident/Sandbag containing zeolite installed before contaminated water started to be transferred
②	Cs Adsorption vessel	Solid (including water)	Metal container containing adsorbent inside(used vessels are stored temporarily outdoor)
④	Shield plug	Solid(detail is unknown)	Shield cover above PCV of Unit 1~3(large amount of Cs-137 released in the accident is trapped between first and second layer of shield plug)
⑥	Cs-137 not included in any of category ①~⑤ neither Cs-137 released to the environment (fuel debris, etc.)	Solid(detail is unknown)	Fuel debris remaining in 1~3 reactor building, etc.

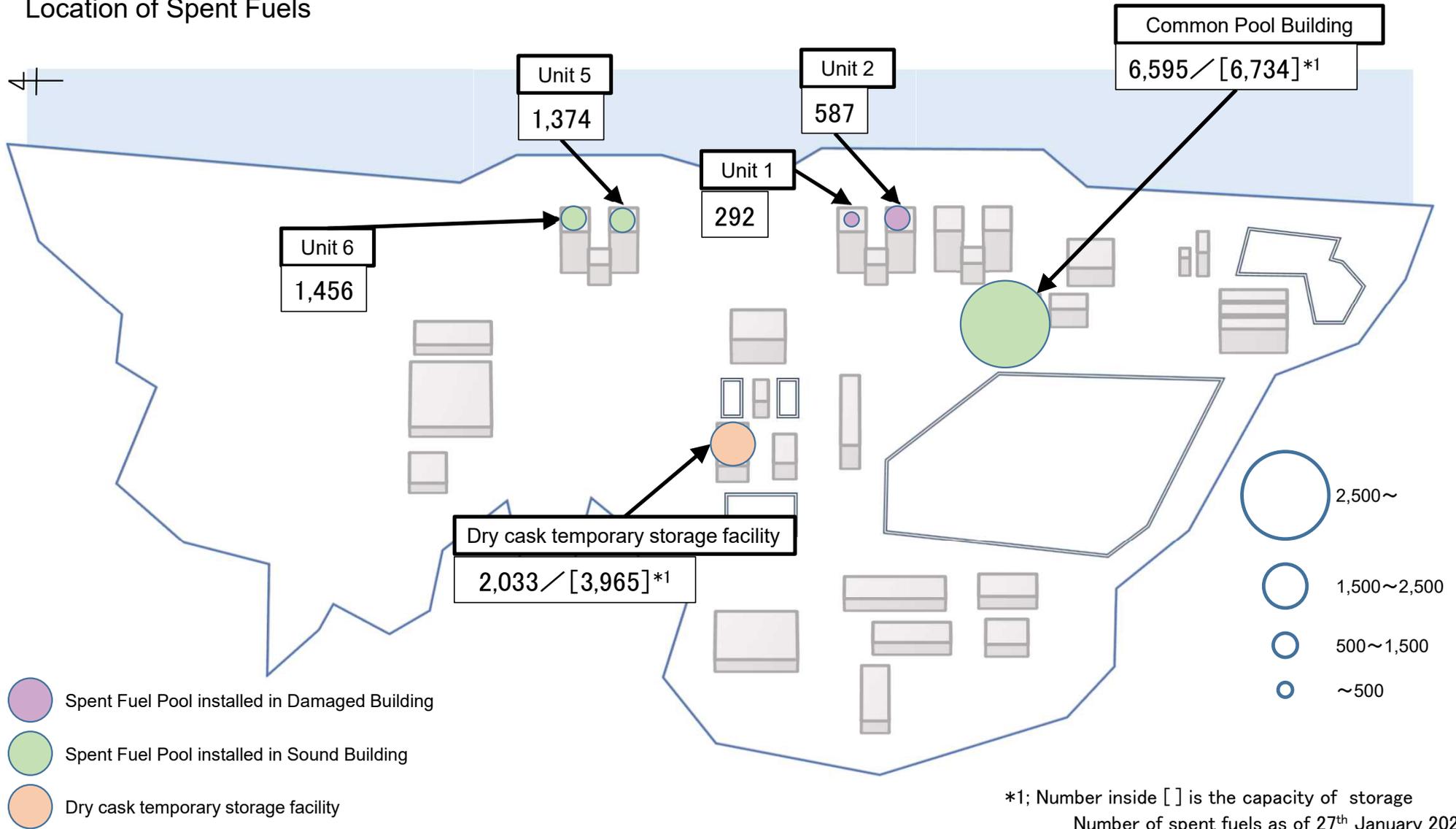
\* : listed up in the ascending order of stability



- Radioactivity decay is considered for 11 years (as of 11<sup>th</sup> March 2022) from the accident, while some data are calculated without considering decay.
- Total amount of Cs-137 in Unit 1~3 is estimated using 「JAEA-DATA/Code2012-2018」 considering decay
- Amount of Cs-137 released to the environment is estimated, referring to 「Additional report of the Japanese government to IAEA—about the TEPCO Fukushima Daiichi Nuclear Power Station accident — (the second report)」 and considering decay
- This material shows the location of Cs-137 except for spent fuels, and for this estimation, the data shown by TEPCO, etc. is used.
- Looking into the secondary waste of water treatment which contains more Sr-90 rather than Cs-137, it is estimated that contained amount of Sr-90 is 44 PBq in HIC, 15PBq in adsorption vessel of Sr, 2PBq in Sludge of decontamination facility, and 1PBq in condensed wastewater.
- Since fraction is rounded up or down. sum of ①~⑥ doesn't match the total amount.

S/C: Suppression Chamber, HIC: High Integrity Container, Sr adsorption vessel: metal container that contains Sr adsorbents, Sludge of Decontamination Facility: sludge and zeolite sandbags generated from decontamination facility, condensed wastewater: wastewater and slurry generated after condensed saline was treated by evaporative concentration facility

# Location of Spent Fuels



## List of Major Inventory (Cs-137)

### Existing in Buildings or adsorption vessels

Location	Inventory (PBq)
Stagnant water (①)	0.2
Unit 3 S/C (⑤)	4
Zeolite (③)	4
Cs adsorption vessel (②)	260
Shield plug (④)	70
Cs-137 not included in any of category ①~⑤ neither Cs-137 released to the environment (fuel debris, etc.)	190
Total amount of Cs-137 released to the environment (atmosphere and ocean) in a few weeks after the accident	14
Total amount of Cs-137 from Unit 1~3	540

### Spent Fuel

Location	Inventory (PBq)
Unit 1 Spent Fuel Pool	130
Unit 2 Spent Fuel Pool	350
Unit 3 Spent Fuel Pool	0
Unit 4 Spent Fuel Pool	0
Unit 5 Spent Fuel Pool	740
Unit 6 Spent Fuel Pool	780
Spent Fuel Common Pool	3,500
Dry Storage Cask	1,100
Total amount	6,600

- ◆ Inventory inside the red frame should be taken measures in high priority
- ◆ Each value above has a large error, because they are evaluated indirectly such as from the balance of the amount of Cs-137 in stagnant water, extrapolation from single data, estimation from the average amount of Cs-137 inside 1 spent fuel assembly, etc.
- ◆ Amount of radioactive material in S/C is listed only for Unit 3, because its analytical data is available.
- ◆ Since fraction is rounded up or down, sum of each inventory doesn't match the total amount.